

Pyrochemical separation of spent nuclear fuel: advances in the frame of the European ACSEPT project

"Actinide reCycling by SEParation and Transmutation"

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ACSEPT program



- > 4.5 years R&D program, from March 2008 to September 2012
- > Funded by the EC within the 7th framework program
- ➤ 12 European countries (+Australia and Japan) 34 partners

Objectives: develop chemical separation processes (hydro+pyro) compatible with fuel fabrication techniques, in view of their future demonstration at the pilot level



S. Bourg, C. Hill, C. Caravaca, C. Rhodes, C. Ekberg, R. Taylor, A. Geiste, G. Modolo, L. Cassayre, R. Malmbeck, M. Harrison, G. de Angelis, A. Espartero, S. Bouvet, N. Ouvrier, Nuclear Engineering and Design (2011)



Pyrochemistry within ACSEPT

Activities in pyrochemistry: 15 partners, 9 countries



ANSTO, Australia EDF, France CEA, France CIEMAT, Spain CNRS, France CRIEPI, Japan ENEA, Italy JRC-ITU, Germany

NLL, UK NRI, Czech Republic PoliMi, Italy PSI, Switzerland Rio Tinto Alcan, France UPMC, France UEDIN, UK

2008: Presentation of the ACSEPT program at the 2008 IPRC meeting (Korea)

2010: ACSEPT International Meeting, Lisbon, Portugal

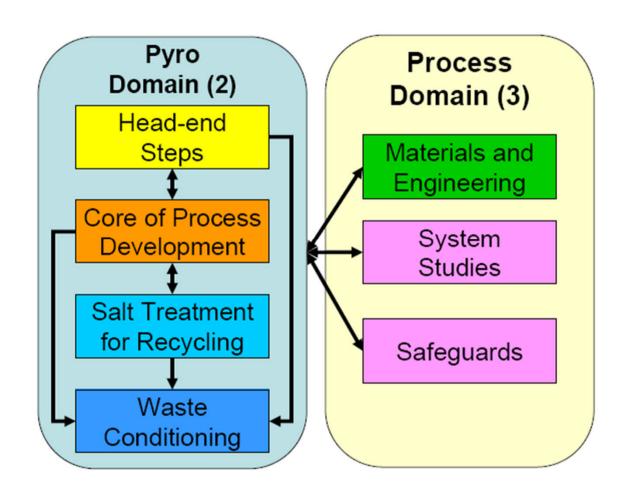
2011: ACSEPT-Korea-Russia exchange meeting, Manchester, UK

2011: ACSEPT-ANL exchange meeting, Goteborg, Sweden





Pyrochemistry within ACSEPT







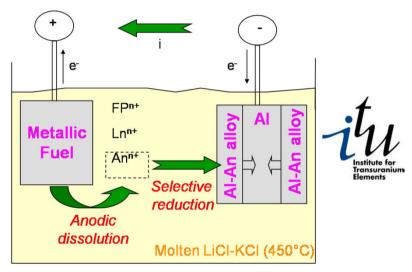
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Core of Process

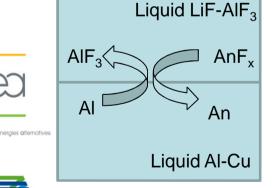
PYROREP+EUROPART = Identification of two reference separation processes: grouped extraction of An dissolved in molten salt by alloying with aluminium

➤ Electrorefining of actinides onto solid aluminium cathode in molten chloride salts (T~450°C)

P. Soucek, R. Malmbeck, C. Nourry, J.-P. Glatz, Energy Procedia (2011)



Principle of the electrorefining



➤ Liquid-liquid reductive extraction in molten fluoride salts/liquid aluminium (T~830°C)

J. Lacquement, H. Boussier, A. Laplace, O. Conocar, A. Grandjean, Journal of Fluorine Chemistry (2009)

Principle of the liquid-liquid extraction



EURATOM

Core of Process

Activities within ACSEPT

- > Electrochemical processes in molten chloride salts
 - maximization of An recovery (exhaustive electrolysis) (ITU,CEA,RTA, CNRS)
 - An-Al alloying properties (ITU,CIEMAT,CEA)

Np-Al: P. Soucek, R. Malmbeck, E. Mendes, C. Nourry, D. Sedmidubsky, J.-P. Glatz, Journal of Nuclear Materials (2009)

Am-Al: G. De Cordoba, A. Laplace, O. Conocar, J. Lacquement, Journal of Nuclear Materials (2009)

Pu-Al: E. Mendes, R. Malmbeck, C. Nourry, P. Soucek, J.-P. Glatz, Journal of Nuclear Materials (2012)

> Electrochemical processes in molten fluoride salts

Investigation of novel separation processes by electrochemical methods (ITU, ANSTO, NRI, CNRS, CEA)

U: C. Nourry, P. Souček, L. Massot, R. Malmbeck, P. Chamelot, J.-P. Glatz, Journal of Nuclear Materials (2012)

N₂: L. Massot, P. Chamelot, M. Gibilaro, L. Cassayre, P. Taxil, Electrochimica Acta (2011)

La: C.P. Fabian, V. Luca, P. Chamelot, L. Massot, C. Caravaca, G.R. Lumpkin, Journal of the Electrochemical Society (2012)

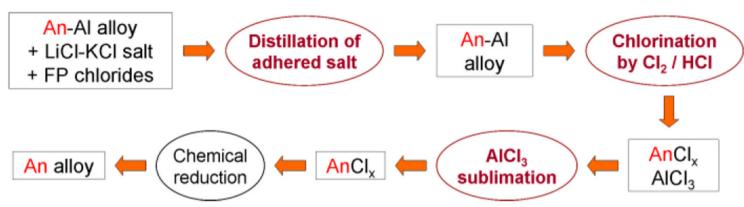
Methods for the recovery of An from Al

Investigation of chemical (chlorination, hydrochlorination) and electrochemical routes, in order to maximize An recovery and recycle AI (CEA, ITU)



Chlorination for An recovery from An-Al alloys





Chlorination route for An recovery

Exp. step	Temp.	Time	Conditions
Distillation	800°C	8 hrs	Vacuum, 4-6·10 ⁻² mbar
Chlorination	150°C	$\begin{array}{c} 2\times 20 \\ \text{hrs} \end{array}$	Cl_2 , molar ratio Cl_2 /alloy = 36
Sublimation	400°C	5 hrs	Ar

Chlorination runs on U-Al and U-Pu-Al

- Optimal T=150°C (no UCl₅/UCl₆ losses)
- But slow kinetics
- Hydrochlorination at 450°C is more efficient



L. Cassayre, P. Soucek, E. Mendes, R. Malmbeck, C. Nourry, R. Eloirdi, J.-P. Glatz, Journal of Nuclear Materials (2011)



Exhaustive electrolysis (//An drawdawn)

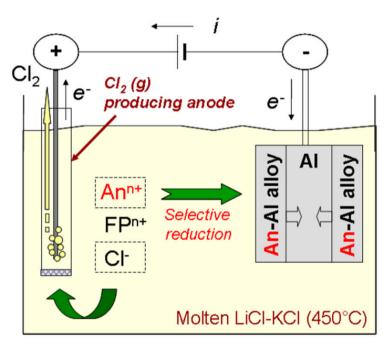
Objectives

FP removal from the electrorefining salt

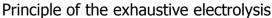
=> Requires An removal in a first step

Principle

- An selective recovery on Al cathode
- Cl₂(g) evolving anode









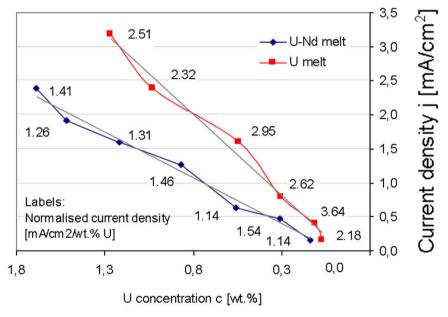




Exhaustive electrolysis

Experimental tests on:

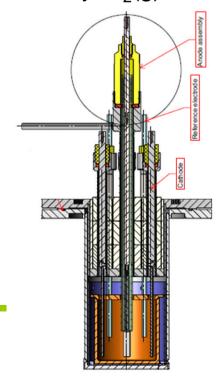
- LiCI-KCI-2.2wt%UCI₃
- LiCI-KCI-1.7wt%UCI₃-1.7wt%NdCI₃



Maximal applicable current densities vs. U concentration

- ⇒ Efficient salt cleaning achieved (U down to 0.07 and 0.14wt%) without Nd codeposition **BUT**
- Low current density ~2mA.cm⁻².wt%(U)⁻¹
- Long electrolysis time
- Corrosion of metallic elements by Cl₂(g)

⇒ Design of a new electrorefiner

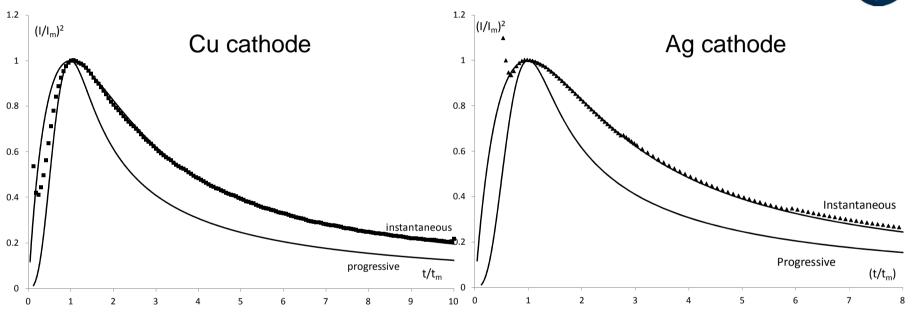






U nucleation model in LiF-CaF₂-UF₃ melts





LiF-CaF₂-UF₃ (3.3 10⁻² mol/kg) on Cu at 840°C, E =-1.27 V/ref. Pt, quasi reference: Pt.

LiF-CaF₂-UF₃ (3.3 10⁻² mol/kg) on Ag at 840°C, E =-1.30 V/ref. Pt, quasi reference : Pt.

=> Instantaneous nucleation

all the nuclei are created at the same time at the beginning of electrolysis





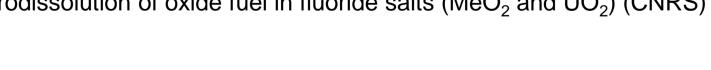
Objectives

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- > Removal of volatile FP
- > Fuel conversion for dissolution/electrodissolution in salts suitable with the core of process

Activities within ACSEPT

- Thermal treatment (PSI)
- Chemical conversion (hydrofluorination) (CEA)
- > Electrochemical conversion
- Direct reduction in chloride salts (Ln oxides) (CIEMAT)
- Direct reduction in fluoride salts (MeO₂ and UO₂) (CNRS)
- Direct electrodissolution of oxide fuel in fluoride salts (MeO₂ and UO₂) (CNRS)



Provides metal phase

suitable for anodic

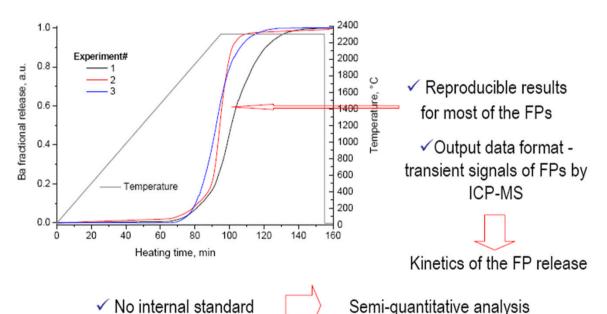
electrodissolution



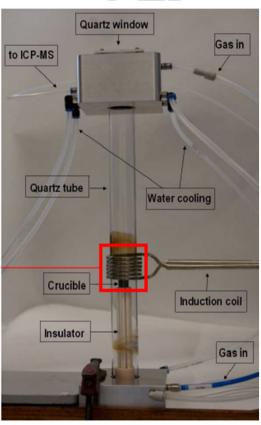
Thermal treatment

Objectives

- Study the release of FPs during the high temperature treatment of SIMFUEL and irradiated fuel
- Modeling the FP release (kinetics and thermodynamics) to support experimental work







High temperature inductive heating device (InVap)



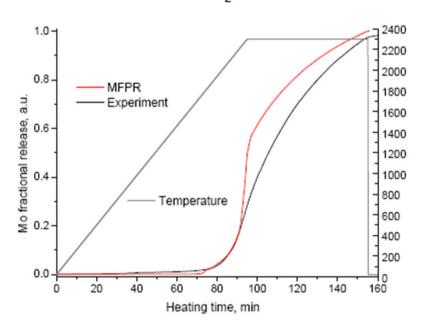
I. Günther-Leopold, N. Kivel, N. Shcherbina, European Winterconference on Plasma Spectrochemistry (2011)



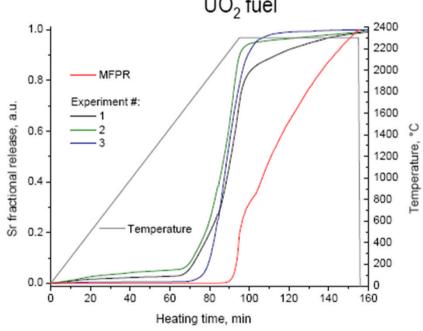


Thermal treatment: modeling vs. experiments

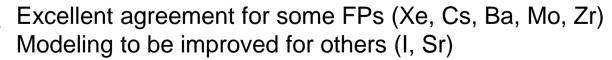
Mo fractional release from irradiated UO₂ fuel



Sr fractional release from irradiated UO₂ fuel











Fuel conversion: direct reduction of UO₂ in LiF-CaF₂

No initial experience in direct reduction

=> Several oxides were tested according to expected increasing difficulty (electronic conductivity, Gibbs energy of formation)

SnO₂: good electronic conductor, Sn liquid at low T

Fe₃O₄: good electronic conductor

TiO, TiO₂: poor conductors, widely studied in chloride melts

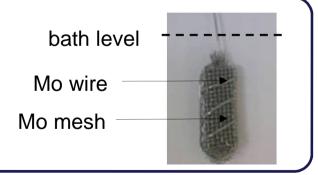
UO₂: poor conductor, radioactive material

 $\Delta_f G^\circ \ll 0$

Anode: Gold spiral

Cathode: oxide in Mo mesh

Salt: LiF-CaF₂ eutectic





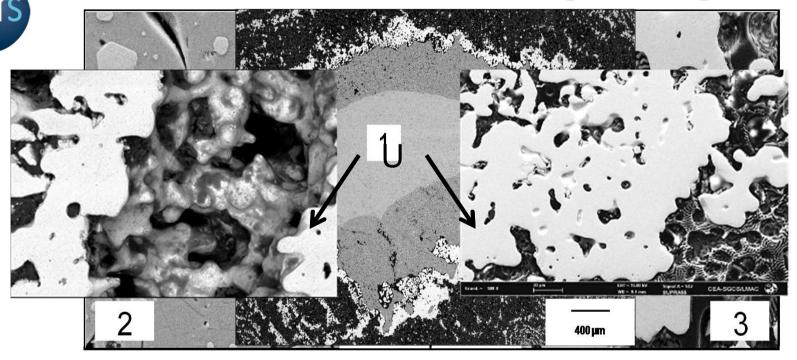
M. Gibilaro, J. Pivato, L. Cassayre, L. Massot, P. Chamelot, P. Taxil, Electrochimica Acta (2011)



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Head-end Steps

Fuel conversion: direct reduction of UO₂ in LiF-CaF₂



Micrographs of cross section of reduced samples: I=-0,15A//-0.30A for t=8000s

- Initial stage of reduction observed at the grain boundary in the partially reduced region with uranium metallic grains
- Full reduction achieved, typical coral-like structure

M. Gibilaro, L. Cassayre, O. Lemoine, L. Massot, O. Dugne, R. Malmbeck, P. Chamelot, Journal of Nuclear Material (2011)



Salt Treatment

Objective

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Treatment of the salts used in the core of process (loaded with Ln and other FP) - minimize wastes; re-use of the salt

Activities within ACSEPT

- ➤ Development of a decontamination process for chloride salts
 - Development of a Zeolite column for FP removal (UK-NNL)
 - Water vapor/Argon sparge precipitation (CEA, CNRS)
 - Test of a liquid lead cathode for removal of Cs, Sr, Ba and Rb (CIEMAT)
 - Development of an aqueous ion-exchange process (ANSTO)
 - Investigation of distillation as technique for decontamination of LiF-AIF₃
 from liquid-liquid reductive extraction process (CEA)
- > Studies of FP behaviour in molten chloride and fluoride salts
 - Speciation (NMR) of La and Cs in oxide containing molten fluorides (CNRS)
 - Development of Molecular Dynamics models in molten salts (UPMC), for salt properties (density, viscosity, conductivity) determination

C. Bessada, O. Pauvert, D. Zanghi, A.-L. Rollet, V. Sarou-Kanian, M. Gobet, G. Moussaed, A. Rakhmatullin, M. Salanne, C. Simon, ECS Transactions (2010)



Salt Treatment

Zeolite ion-exchange



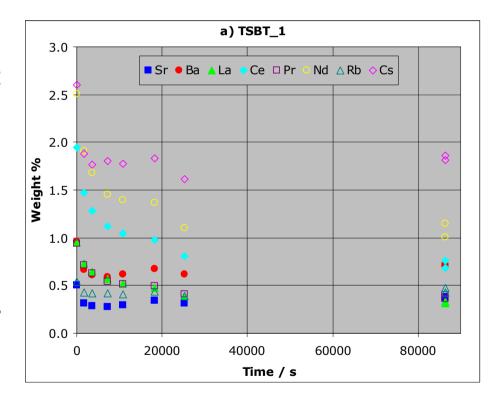
 Reliable decontamination factors determined for range of fission products Agree with expectations, i.e. RE > AE

Element	Nd ³⁺	Ce ³⁺	Sm ³⁺	Eu ³⁺	Eu ²⁺	Sr ²⁺	Ba ²
DF	10.0	8.0	6.7	2.5	2.9	2.2	1.8

Reference Waste Salt experiments:
 Competition effects between +3 (Ln) and +2 species (Sr, Ba)
 Alkali metals (Cs, Rb) rapidly reach a constant concentration, whereas the La,
 Ce, Pr and Nd all continue to ion-exchange

The degree of ion-exchange depends on the starting concentrations of all the species present.

over the full 24 hours of the experiment.





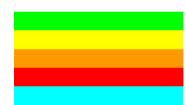


Salt Treatment

Comparison of processes



	Lanthanides	Alkaline Earths	Alkali Metals
Salt Decontamination Process	(La, Ce, Nd, Pr)	(Sr, Ba)	(Cs, Rb)
Ion-exchange (zeolites)			
Hybrid aqueous closed loop			
Precipitation (carbonates/phosphates)			
Precipitation (oxygen sparging)			
Precipitation (wet argon sparging)			
Electrolysis			
Li-Reduction			
Melt crystallisation			



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Fully effective, demonstrated at lab-scale (>90% efficient)
Partially effective, demonstrated at lab-scale (10-90% efficient)
Slightly effective, demonstrated at lab-scale (<10% efficient)
Ineffective, demonstrated at lab-scale (0% efficient)
Assumed effective, but not fully demonstrated

Salt Decontamination Process	Advantages	Disadvantages
Ion-exchange (zeolites)	Removes all FPs.	Low DFs for AMs and AEs High waste volume.
Hybrid aqueous closed loop	Removes all FPs.	Limited experimental data.
	Wasteform contains no chloride.	Not fully demonstrated.
		Complex flowsheet.
		Re-conditioning of salt required.
Precipitation (wet argon sparging)	Effective for Ln's.	Not fully demonstrated.
		Combination of water and melted salt.
Electrolysis	Effective for Ln's	Ineffective for AMs & AEs.
		Evolution of chlorine.



Waste Conditioning

ObjectiveS

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Conditioning of chloride, fluoride and metallic wastes arising from pyro processes

Activities within ACSEPT

- > Assessment of sodalite as candidate matrix for spent chloride salt confinement
 - Synthesis of Li-K sodalite (100 g scale) (ENEA)
 - Validation of pure Li-K sodalite as a potential matrix (CEA, PoliMi)
- > Chloroapatite as an alternative matrix for spent chloride salt confinement
 - Synthesis of a chloroapatite phase for the incorporation of alkaline, alkaline earth and rare earth elements (CEA)
- Confinement of metallic fission products
 - Immobilization of noble FP (Pd, Mo, Tu, Rh) by formation of metallic solid solutions (Cu-Ni, Cu-Sn, Al)(CEA)



Waste Conditioning

Ceramic processing



Sodalite based (Na₈Al₆Si₆O₂₄Cl₂) ceramic processing with Hot Uniaxial Press (HUP) or Pressureless Consolidation (PC)

G. De Angelis presentation



Chloroapatite Ca₁₀(PO₄)₆Cl₂

 $3 Ca_3(PO_4)_2 + CaCl_2 \rightarrow Ca_{10}(PO_4)_6Cl_2$: calcination at 850°C/3x12h

➤ Chlorospodiosite Ca₂PO₄Cl

 $Ca_3(PO_4)_2 + CaCl_2 \rightarrow 2 Ca_2(PO_4)Cl$: calcination at 750°C/3x15h

- Sintering by HUP
- Process optimisation (T, P, duration): densification ratios > 92%





Waste Conditioning

Leaching tests

- > Static leaching tests:
 - Contact time (1-7-15-30-90-150 days)
 - Temperature (25°C and 90°C)
- Dynamic Soxhlet tests at 100°C for 0.3-1-3-7-10 days

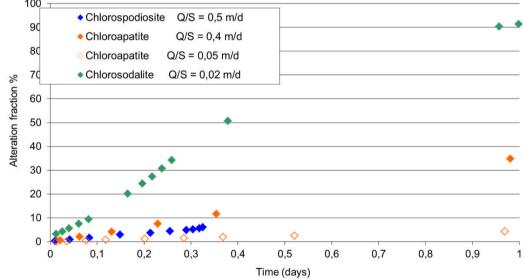




Sodalite phase: significant release of chloride salt, high initial dissolution rate at 100°C in pure water, and holes in the matrix after 150 days leaching at 90°C.

⇒sodalite phase cannot be retained as a suitable matrix

Better behaviour of chloroapatite – to be confirmed with long term tests



Comparison of the alteration fraction (NL x Sspec, %) of chloroapatite, chlorospodiosite and chlorosodalite versus time





Concluding remarks

- ACSEPT final meeting will take place at the end of the Atalante meeting (Montpellier, France)
- ➤ Following Fukushima accident, EC policy regarding nuclear fission research programs has changed: new programs focused on safety of existing reactors, no new developments (e.g. GENIV) except for An burning (double strata concept)
- ➤ However, the SACSESS (Safety of ACtinide SEparation proceSSes) program will be funded by EC, starting in 2013
 - Pyro activities will be focused on treatment of ADS matrices (cercer and cermet), online monitoring and basic data acquisition

Thank you for your attention!

